# Assessment of correlations and models for the prediction of CHF in water subcooled flow boiling

G. P. CELATA, M. CUMO and A. MARIANI

ENEA, Thermal Process Engineering Division, Via Anguillarese, 301, I-00060 Rome, Italy

(Received 23 March 1993 and in final form 25 June 1993)

Abstract—The present paper provides an analysis of available correlations and models for the prediction of Critical Heat Flux (CHF) in subcooled flow boiling in the range of interest of fusion reactors thermal-hydraulic conditions, i.e. high inlet liquid subcooling and velocity and small channel diameter and length. The aim of the study was to establish the limits of validity of present predictive tools (most of them were proposed with reference to LWR thermal-hydraulic studies) in the above conditions. The reference dataset represents almost all available data (1865 data points) covering wide ranges of operating conditions in the frame of present interest (0.1 MPa; <math>0.3 < D < 25.4 mm; 0.1 < L < 0.61 m; 2 < G < 90.0 Mg m<sup>-2</sup> s<sup>-1</sup>;  $90 < \Delta T_{sub,in} < 230$  K). Among the tens of predictive tools available in literature four correlations (Levy, Westinghouse, modified-Tong and Tong-75) and three models (Weisman and Ileslamlou, Lee and Mudawar and Katto) were selected. The modified-Tong correlation and the Katto model seem to be reliable predictive tools for the calculation of the CHF in subcooled flow boiling.

# INTRODUCTION

As IS KNOWN [1-4], the removal of high heat fluxes as required by fusion reactor thermal-hydraulics, may be achieved by making use of highly subcooled water flow boiling at high liquid velocity. As successful use of this technique requires the critical heat flux (CHF) to be avoided, it is necessary, in addition to the availability of experimental data for the understanding of the phenomenon, to have CHF prediction tools for calculation and design purposes. Scarcity of experimental data in the range of interest for fusion reactor thermal-hydraulics implies also a lack of suitable correlations for the prediction of subcooled CHF. Available correlations for the prediction of the CHF in subcooled flow boiling were recommended in ranges of pressure, liquid velocity and subcooling, and consequently heat fluxes, typical of the Light Water Reactors (LWR), i.e. much different than those required for the cooling of the high heat flux components of the fusion reactor. In the case of LWRs the heat flux to be removed is, as an order of magnitude, around 1 MW  $m^{-2}$ , while in the case of fusion reactors some components may require heat fluxes up to 60 MW  $m^{-2}$  to be removed. Nonetheless, it is known that correlations cannot be used in a reliable way outside the range recommended by authors. On the other hand, the absence of suitable experimental data and visual information that could clarify the basic mechanisms of subcooled flow boiling under these conditions does not allow a full understanding of the phenomenon so to enable an actual mechanistic description of the phenomenon itself in a model.

The aim of the present paper is to provide an assessment of what is available in the literature, both in

terms of correlations and in terms of models, against most of the experimental data existing in the literature in the range of interest of fusion reactors. The result is the definition of the bounds of reliability of the existing prediction tools. To this purpose, among the tens of correlations available, results are reported here only for those providing a consistent prediction of most of the data set, namely Levy [5], Westinghouse [6], modified-Tong [7] and Tong-75 [8] correlations. The three existing models proposed by Weisman-Ileslamlou [9], Lee-Mudawar [10] and Katto [11] were reported. It must be pointed out here that the above correlations (with the only exception of the modified-Tong correlation) and models (with the exception of the Katto model) are tested in the present study against experimental data whose operating ranges are outside the recommended ones. Therefore, possible inaccuracy of prediction is not to be ascribed to a weakness of the single correlation or model, but only to the misuse of them (out of range). The purpose of their use in the present comparative study is only to check the possibility of extending the validity range of existing correlations and models to have predictive tools in the range of interest of fusion reactor thermal hydraulics. As far as data sets are concerned [12-36] they cover the following operating ranges: 0.1 < p< 8.4 MPa; 0.3 < D < 25.4 mm; 0.1 < L < 0.61 m;  $2 < G < 90.0 \text{ Mg m}^{-2} \text{ s}^{-1}$ ;  $90 < \Delta T_{\text{sub,in}} < 230 \text{ K}$ .

## AVAILABLE EXPERIMENTAL DATA

Subcooled flow boiling CHF was extensively studied in the past [37–39] in a range of interest for LWRs, that refers to conditions very far from fusion reactor thermal-hydraulics requirements (heat flux up to 60

NOMENCLATORE								
Bo	boiling number, $q''/G\lambda$	X	thermal equilibrium quality.					
С	parameter defined in (3), (4) and (6)							
CHF	critical heat flux [W m <sup>-2</sup> ]	Greek sys	mbols					
$C_{\rm p}$	specific heat [J kg <sup>-1</sup> K <sup>-1</sup> ]	α	void fraction					
D	channel diameter [mm]	$\delta$	liquid sublayer thickness [µm]					
$D_0$	reference diameter, $D_0 = 0.0127$ m	λ	latent heat [J kg <sup>-1</sup> ]					
$D_{c}$	test section equivalent internal diameter	μ	dynamic viscosity [kg m $^{-1}$ s $^{-1}$ ]					
	[mm]	ρ	density [kg m <sup>-3</sup> ]					
F	parameter defined in (1)	σ	surface tension [N m <sup>-1</sup> ]					
$f_0$	parameter defined in (7)	Ψ	parameter defined in (6).					
G	mass flux [kg m <sup><math>-2</math></sup> s <sup><math>-1</math></sup> ]							
g	gravitational acceleration [m s <sup>-2</sup> ]	Subscript	s					
h	enthalpy [J kg <sup>-1</sup> ]	b	pertains to bulk conditions					
$h_1$	heat transfer coefficient [W m <sup>-2</sup> K <sup>-1</sup> ]	CHF	pertains to burnout conditions					
Ja	Jacob number, $C_{\rm p}(T_{\rm b}-T_{\rm sub})\rho_{\rm f}/\lambda\rho_{\rm g}$	conv	convective					
K	thermal conductivity $[W m^{-1} K^{-1}]$	crit	critical (thermodynamic) conditions					
k	velocity coefficient	ex	exit					
L	channel length [cm]	ſ	pertains to the liquid in saturated					
$L_{\rm B}$	vapour blanket length [ $\mu$ m]		conditions					
Pr	Prandtl number, $C_p \mu/K$	g	pertains to the vapour					
р	pressure [MPa]	in	inlet					
q''	heat flux	1	pertains to the liquid					
Re	Reynolds number, $GD/\mu$	out	outlet					
$T, \Delta T$	temperature, temperature difference	pb	pool boiling					
	[°C. K]	sat	saturated conditions					
$U_{\mathbf{B}}$	vapour blanket velocity [m s <sup>1</sup> ]	sub	subcooled conditions					
$U_\delta$	liquid sublayer velocity as $\delta$ [m s <sup>-1</sup> ]	w	pertains to the wall.					

# 

MW m<sup>-2</sup>, mass flux up to 40 Mg m<sup>-2</sup> s<sup>-1</sup>, inlet subcooling up to 200 K, pressure up to 5.0 MPa, L/Dfrom 10 to 200). In the recent past several researches were initiated to achieve a deeper understanding of subcooled flow boiling process.

Experimental points which will be used here to assess the available predictive tools are those presented by Boyd [12-14], Inasaka and Nariai [15], Nariai et al. [16], Achilli et al. [17], Celata et al. [18-21], Gambill and Greene [22], Vandervort et al. [23], Loosmore and Skinner [24], Ornatskii and Vinyarskii [25], Ornatskii and Kichigan [26], Ornatskii [27]. Knoebel et al. [28], Mirshak et al. [29], Babcock [30], Burck and Hufschmidt [31], Mayersak et al. [32]. Schaefer and Jack [33], Thorgerson [34], Zeigarnik et al. [35], and Gambill et al. [36]. The ranges of operating conditions for the data considered to establish correlations and models are summarized in Table 1, while a graphic representation of the overall operating range is given in Fig. 1 (0.1MPa; 0.3 < D < 25.4 mm; 0.0025 < L < 0.61 m; 2 < G < 90.0 Mg m<sup>-2</sup> s<sup>-1</sup>;  $90 < \Delta T_{\text{sub.in}} < 230$  K). The total number of data points used in the present study is 1865.

#### **AVAILABLE CORRELATIONS**

A rich review of correlations for subcooled flow boiling CHF was given by Boyd [4], and many of them were tested by Celata [1, 40] and by Nariai and Inasaka [41]. Among the tens of correlations tested in [1, 40, 41], attention has been paid here only to those providing a consistent prediction of experimental data, so as to make possible an extension of their validity bounds. The correlations considered in the present calculations are: Levy [5], Westinghouse [6], modified-Tong [7] and Tong-75 [8] correlations.

 $q_{\rm conv}''$ 

$$q_{CHF}'' = q_{pb}'' + q_{conv}'' + F \qquad (1)$$

$$q_{pb}'' = 0.131 \lambda \rho_g \left[ \frac{\sigma g^2 (\rho_1 - \rho_g)}{\rho_g^2} \right]^{1/4}$$

$$= 0.696 (K \rho_1 C_p)^{1/2} \left( \frac{\rho_1 - \rho_g}{\sigma} \right)^{1/4} \times \left[ \frac{\sigma g^2 (\rho_1 - \rho_g)}{\rho_g^2} \right]^{1/8} \Delta T_{sub}$$

$$F = h_1 (T_w - T_{sat}) + h_1 \Delta T_{sub}$$

$$T_w - T_{sat} = \frac{60}{e^{\rho/900}} \left( \frac{q''}{10^6} \right)^{1/4}$$

$$h_1 = 0.023 \frac{K}{D} R e^{0.8} P r^{1/4}$$

CHF Data [ref.]	No. of points	$T_{in}$ [°C]	p [MPa]	D [mm]	L [mm]	G [Mg m <sup>-2</sup> s <sup>-1</sup> ]	$q_{\rm CHF}^{\prime\prime}$ [MW m <sup>-2</sup> ]
Celata et al. [18]	43	18.6-54.6	0.1-2.2	2.5, 4, 5	100	2.2-32.6	4.0-42.7
Celata et al. [19]	88	29.8-70.5	0.6-2.6	2.5	100	10.1-40.0	12.1-60.6
Celata et al. [21]	48	29.3-71.5	0.5-2.6	4.0	100	5.0-40.0	10.6-54.4
Celata et al. [21]	7	29.3-40.7	0.8	5.0	100	4.1-20.0	13.0-34.7
Celata et al. [20]	14	29.8-75.9	2.1 - 5.0	6.0	100	5.0-10.0	11.8-27.8
Celata et al. [20]	46	29.1-80.7	0.4-5.0	8.0	100-150	2.0-10.0	7.4-29.5
Inasaka-Nariai [15]	29	25.0-78.0	0.3-1.1	3.0	100	4.3-30.0	7.3-44.5
Nariai et al. [16]	95	15.4-64.0	0.1	1.0-3.0	10-100	6.7-20.9	4.6-70.0
Boyd [12–14]	10	20.0	0.77-1.66	3.0	289.7	4.4-40.5	6.0-41.5
Achilli et al. [17]	35	26.4-158.2	1.0-5.5	8.0-15.0	150-300	4.6-14.9	11.0-35.6
Gambill-Greene [22]	7	4.9-35.8	0.1	7.8	45-157	13.0-26.0	15.8-33.0
Vandervort et al. [23]	210	6.4-84.9	0.1-2.3	0.3-2.6	2.5-66	8.4-42.7	18.7-123.8
Loosmore-Skinner [24]	202	3.2-130.9	0.1-0.7	0.6-2.4	6.3-150	3.0-25.0	6.7-44.8
Ornatskii–Vinyarskii [25]	125	6.7–155.6	1.1-3.2	0.4 - 2.0	11.2-56	10.0-90.0	27.9-227.9
Ornatskii–Kichigan [26]	117	2.7-204.5	1.0-2.5	2.0	56	5.0-30.0	6.4-66.6
Ornatskii [27]	68	1.5-153.7	1.0-3.2	0.5	14	20.0-90.0	41.9-224.5
Knoebel et al. [28]	376	0.3-104.8	0.2-0.7	9.5	610	3.9-13.7	3.3-11.4
Mirshak et al. [29]	56	5.9-68.7	0.2-0.6	6.0-11.9	489-610	4.7-12.2	3.9-10.0
Babcock [30]	57	19.9-242.7	0.4-8.4	7.9–25.4	610	2.4-11.4	4.9-11.8
Burck-Hufschmidt [31]	143	16.7-60.8	1.1-3.1	10.0	350	0.9-3.8	4.5-12.2
Mayersak et al. [32]	1	18.0	2.9	11.7	585	44.4	42.8
Schaefer-Jack [33]	2	15.6-18.9	1.3-1.5	3.05	19	61.2-61.7	125.0-130.0
Thorgerson [34]	42	1.1-79.2	0.45	7.8-8.4	610	4.2-13.4	4.2-12.4
Zeigarnik et al. [35]	21	0.6-134.1	0.5-3.0	4.0	250	4.8-20.6	9.4-32.6
Gambill et al. [36]	23	8.8-23.9	0.10.5	3.2-7.8	37-416	7.0-53.0	7.0-48.7
Total	1865	0.3-242.7	0.1-8.4	0.3-25.4	2.5-610	0.9-90.0	3.3-227.9

Table 1. Experimental data used for present calculation

recommended in the ranges 0.6 < G < 11 Mg m<sup>-2</sup> s<sup>-1</sup>; 0.4 MPa; <math>2 < D < 12 mm.

Westinghouse [6]:

$$q_{CHF}^{\sigma} = (0.23 \times 10^{6} + 0.094G)(3 + 0.01\Delta T_{sub}) \times [0.435 + 1.23 \exp(-0.0093L/D)]^{*} \times \left\{ 1.7 - 1.4 \exp\left[-0.532\left(\frac{h_{sat} - h_{in}}{\lambda}\right)^{3/4}\left(\frac{\rho_{g}}{\rho_{f}}\right)^{-1/3}\right] \right\}$$
(2)

recommended in the ranges 0.3 < G < 11 Mg m<sup>-2</sup> s<sup>-1</sup>,  $5.7 MPa, <math>1.25 < q''_{CHF} < 12.5$  MW m<sup>-2</sup>,  $0 < \Delta T_{sub} < 126$  K.

Tong [7]:

$$\frac{q_{\rm CHF}'}{\lambda} = C \frac{G^{0.4} \mu_{\rm f}^{0.6}}{D^{0.6}}$$
(3)

$$C = 1.76 - 7.433x_{\rm ex} + 12.222x_{\rm ex}^2 \tag{4}$$

where  $\lambda$  is the latent heat and  $\mu_{\rm f}$  is the dynamic vis-



FIG. 1. Ranges of operating conditions for the data used in present calculations.

cosity of saturated liquid (SI units). The Tong correlation may be also presented in the form :

$$Bo = \frac{C}{Re^{0.6}} \tag{5}$$

where *Bo* and *Re* are Boiling number and Reynolds number, respectively. This correlation is often known as Tong-68 correlation. We modified the parameter *C*, together with a slight modification of the Reynolds number power, to give a more accurate prediction in the range of pressures below 5.0 MPa, as the Tong correlation was recommended for pressures higher than 7.0 MPa. The modification of the Tong correlation was based on data reported in refs. [18]– [21] (2.2 < *G* < 40 Mg m<sup>-2</sup> s<sup>-1</sup>, 0.1 < *p* < 5.0 MPa, 2.5 < *D* < 8.0 mm, 12 < *L/D* < 40, 15 <  $\Delta T_{sub.ex}$ < 190 K, 4.0 <  $q''_{CHF}$  < 60.6 MW m<sup>-2</sup>). The new expression of the Tong correlation is

$$Bo = \frac{C}{Re^{0.5}} \tag{6}$$

with

 $C = (0.216 + 4.74 \times 10^{-2}p)\Psi [p \text{ in MPa}]$   $\Psi = 0.825 + 0.986x_{ex} \quad \text{if } x_{ex} > -0.1;$   $\Psi = 1 \quad \text{if } x_{ex} < -0.1$   $\Psi = 1/(2 + 30x_{ex}) \quad \text{if } x_{ex} > 0$ (exit saturated conditions).

Tong-75 [8]:

$$q_{\rm CHF}'' = 0.23 f_0 G \lambda \left[ 1 + 0.00216 \left( \frac{p_{\rm ex}}{p_{\rm crit}} \right)^{1.8} R e^{0.5} J a \right]$$
(7)

2

where

$$f_0 = \frac{8\left(\frac{D_{\rm e}}{D_0}\right)^{0.3}}{Re^{0.6}}$$

with  $D_0 = 1.27 \times 10^{-2}$  m

$$Re = \frac{GD}{\mu_{\rm f}(1-\alpha)}$$

with  $\alpha$  evaluated by using Thom's correlation [42];

$$Ja = \frac{C_{\rm p}(T_{\rm b} - T_{\rm sub})}{\lambda} \frac{\rho_{\rm f}}{\rho_{\rm g}}$$

recommended in the ranges 7.0 MPa,<math>3 < D < 10 mm, 5 < L/D < 100, 0.7 < G < 6.0 Mg m<sup>-2</sup> s<sup>-1</sup>,  $-1.0 < x_{in} < 0.0$ .

#### AVAILABLE MODELS

As is known, models have the advantage, with respect to correlations, of being able to characterize not only the existing and developing data base, but also to be used to predict CHF beyond the established data base. In this sense visual information, not available so far in detail, would be of great help for a full understanding of the basic mechanisms of CHF in subcooled flow boiling at high liquid velocity and inlet subcooling, enabling the development of a mechanistic model of CHF more closely to reality. Anyway, at the moment, three different models are available in the literature for the prediction of the CHF in subcooled flow boiling: the Weisman and Ileslamlou [9], the Lee and Mudawar [10] and the Katto [11] models. The Weisman and Ileslamlou model [9] (extension of the Weisman and Pei model [43]) is based on the existence of a bubbly layer adjacent to the heater surface. At the CHF, the bubbles agglomerate into a vapour blanket that prevents the liquid core from cooling the heater wall. It assumes that the turbulent interchange at the outer edge of the bubbly layer is the limiting mechanism. The void fraction in the bubbly layer is determined by a balance between the outward vapour flow away from the wall and the inward liquid flow at the bubbly layer-core interface. They postulated that CHF occurs when the void fraction in the bubbly layer just exceeds the critical value of 0.82. The void fraction was calculated under the assumption of homogeneous two-phase flow in the bubbly layer. With reference to the previous model description the new one accounts for high subcooling condition effects (energy balance at the bubbly layer-core interface) making the computation of the CHF an entirely local calculation (authors claim that under subcooled and low quality conditions CHF is a local phenomenon). Authors tested and assessed their model within the following parameters ranges:  $-0.12 \ge x_{ex} \ge -0.46$ ; p = 6.8-19 MPa; D = 1.9-37.5 mm; L = 76-1950 mm; G = 1.3-10.5 $Mg m^{-2} s^{-1}$ 

The Lee and Mudawar model [10] (liquid sublayer dryout model) is a mechanistic CHF model based on the observation that, during fully developed boiling. a vapour blanket forms in the vicinity of the heated wall by the coalescence of small bubbles, leaving a thin liquid sublayer in contact with the heated wall beneath the blanket. The onset of sublayer dryout was assumed to be triggered by a Helmholtz instability at the sublayer-vapour blanket interface, and CHF was postulated to occur when the rate of heat supplied at the wall exceeds the enthalpy of fresh liquid entering the sublayer from the bubbly layer and core regions (or, in other terms, when the rate of sublayer mass loss by evaporation exceeds that of the liquid entering the sublaver from the core region). Although the model is mechanistic in nature, describing a specific process associated with CHF, its development requires the use of available correlations to describe the dynamics of bubbles in the wall region. The model was assessed by the authors (choice of correlations) on the following ranges of parameters: p = 5-17.6MPa; G = 1-5.2 Mg m<sup>-2</sup> s<sup>-1</sup>; D = 4-16 mm;  $\Delta T_{\rm sub} = 0-59$  K. The two models reported above were proposed by respective authors for high pressure conditions. In particular, the Lee and Mudawar model was developed for high pressure conditions only since it assumes the existence of a vapour layer in a small wall region while maintaining a velocity profile in the core liquid which can be represented by the law of the wall. This condition is simply not valid for low pressure systems and the model is, therefore, not expected to yield accurate CHF predictions at low pressure. Similar premises could be forwarded for the Weisman and Ileslamlou model. As already stated in the introduction, they are applied here for low pressure conditions just to check their performances in this operating ranges. In particular, the Lee and Mudawar model looks very attractive for its mechanistic nature and can be considered as a starting point to obtain a model whose validity could be extended to low pres-

sure conditions. The Katto model [11] is based on the same mechanism as the Lee and Mudawar model, from which it borrows much of the original derivation, i.e. liquid sublayer dryout mechanism. A thin vapour layer or slug (called 'vapour blanket') is formed, due to accumulation and condensation of the vapour furnished from the wall, overlying a very thin liquid sublayer adjacent to the wall. CHF is assumed to occur when the liquid sublayer is extinguished by evaporation during the passage time of the vapour blanket sliding on it. Parameters to be determined in the description of the mechanistic model by Katto are : initial thickness of the sublayer,  $\delta$ , vapour blanket length,  $L_{\rm B}$ , and velocity,  $U_{\rm B}$ . The evaluation of  $\delta$  is obtained differently from Lee and Mudawar model using a non-dimensional correlation derived in a previous study of CHF in pool boiling [44]. Vapour blanket length  $L_{\rm B}$  is set equal to the critical wavelength of Helmholtz instability of the liquid-vapour interface (same as in the Lee and Mudawar model). Vapour blanket velocity  $U_{\rm B}$  is evaluated by relating it to the local velocity  $U_{\delta}$  of the near wall two-phase flow (which is assumed to be homogeneous flow) at a distance  $\delta$  from the tube wall.  $U_{\delta}$  is evaluated by the Karman velocity distribution and  $U_{\rm B}$  is set equal to  $kU_{\delta}$ , where k is called the velocity coefficient and is the only one quantity to be determined empirically in the Katto model. The velocity coefficient k (nondimensional correlation as a function of Reynolds number, liquid and vapour density, and void fraction) was derived on data-sets published in refs. [12, 15, 18, 45-46], practically transforming the model in an empirical correlation. The Katto model results tested on the following range of parameters (water): D = 1.14-11.07 mm; p = 0.1-19.6 MPa;  $G = 0.35-40.6 \text{ Mg m}^{-2} \text{ s}^{-1}$ ;  $\Delta T_{\text{sub,out}} = 0-117.5 \text{ K}$ .

#### RESULTS

A summary of present calculations is reported in Tables 2-4, where the performances of correlations and models used are given. Table 2 reports the performances of correlations and models in terms of maximum deviation from the experimental value, i.e. the percentage of data points predicted within a given error band. A graphic representation of these results is shown in Fig. 2, where the percentage of data points predicted within a given error band is plotted against the error band. The best statistics are provided by the modified-Tong correlation and Katto models that both predict the CHF accurate to 25% for about 75% of the time (76.5% for modified-Tong correlation and 72.3% for Katto model). They show a very similar behaviour, even though the modified-Tong correlation almost always exhibits a higher percentage of points predicted within the fixed error band. It must be pointed out that the Katto model, contrarily to all other correlations and models, is not able to calculate all the data points. It fails for 950 points out of 1865 (50.9%) that are discarded for the reason that the calculation procedure of the Katto model requires a void fraction in the boiling layer less than 0.7. This condition is matched whenever the inlet subcooling is medium/low and is associated with low velocity or low mass flow rate. This is the limit considered by the author for the validity of the assumption of homogeneous flow in the two-phase boundary layer. For conditions where a higher void fraction is predicted in the calculation procedure the model fails the evaluation of the CHF. Among the other three correlations (Levy, Westinghouse and Tong-75) Westinghouse correlation provides a fairly good prediction, even though below the performance of modified-Tong correlation. Tables 3 and 4 report the r.m.s. error for correlations and models, respectively, and for each data-set analyzed, besides the total values. A graphic representation of these results is shown in Fig. 3, where the r.m.s. error is plotted vs the different datasets employed. Globally the best behaviour is exhi-

Table 2. Performance of correlations and models in term of maximum deviation from the experimental value

Correlation model	% of points within $\pm 15\%$	% of point within $\pm 25\%$	% of points within $\pm$ 30%	% of points within <u>+</u> 50%	Calculated points
Katto	51.8	72.3	80.1	95.7	915
Lee-Mudawar	3.14	6.5	8.8	20.1	1847
Weisman-Ileslamlou	17.8	30.9	37.1	59.9	1864
Tong-75	30.2	47.3	52.9	65.8	1865
Westinghouse	40.1	65.4	74.8	93.8	1865
Levy	22.8	37.7	43.5	61.3	1859
modTong	55.0	76.5	82.7	98.1	1865



FIG. 2. Percentage of data points predicted within a given error band vs the error band.

bited by the modified-Tong correlation that shows a global r.m.s. error of 21.2%. A slightly higher global r.m.s. error, 24.5%, is given by the Katto model, that provides the best prediction among models (with the limitations on the total number of calculated points as above). From Fig. 3 and Tables 3 and 4 it is also possible to observe the behaviour of used correlations and models for each different reference data-set. Modified-Tong correlation shows a better r.m.s. error than Katto model for seventeen data-sets out of twenty-four.

Apart from the pure statistics, that may be of some help to establish the merits of predictive tools but are certainly not exhaustive for a comprehensive analysis of them, results of calculated vs experimental CHF values are plotted in Fig. 4 for the four correlations and in Fig. 5 for the three models. Figures 6–12 report, for each correlation and model, the ratio beween calculated and experimental CHF vs the main thermalhydraulics and geometric parameters, i.e. mass flux, pressure, channel diameter and channel length. Among the correlations, as expected by the above statistics, the most homogeneous behaviour is provided by the modified-Tong correlation that, although assessed on ENEA data [18-21], is able to give a good prediction of all the other data-sets. Only a few data from Ornatskii et al. [25-27] in the range 40-75 MW  $m^{-2}$  are overpredicted above the average value (small diameters). The Westinghouse and the Levy correlations give an underprediction of Nariai et al. data [16] (atmospheric pressure and very small tube diameters) and Vandervort et al. data [23] (very small tube diameters). The Westinghouse correlation also overpredicts the Gambill-Greene data [22] (atmospheric pressure), while the Levy correlation overpredicts most of Boyd data [12-14] and underpredicts the very high CHF data of Ornatskii et al. [25-27]. The Tong-75 correlation essentially fails in the prediction of Gambill-Greene, and Nariai et al. datasets, both at atmospheric pressure, and Ornatskii et al. data sets. For the data-sets not mentioned, all the three correlations are able to predict most of the data



FIG. 3. Calculated r.m.s. errors vs data-sets.

within  $\pm 25\%$ . Being developed for LWR conditions, the above correlations were proposed for tube diameters above 2–3 mm and pressures higher than 4.0– 5.0 MPa. Their failure was therefore expected when predicting experimental data characterized by very low pressure and/or very small tube diameter. It is instead surprising that the mass flux, considerably higher than the upper limit recommended for the above correlations (11 Mg m<sup>-2</sup> s<sup>-1</sup>), does not show, globally, a dramatic systematic effect such as the pres-

Correlation	Levy		Tor	Tong-75		Westinghouse		modTong	
CHF Data	r.m.s.		r.m.s.		r.m.s.		r.m.s.		
[ref.]	[%]	Points	[%]	Points	[%]	Points	[%]	Points	
Total	59.5	1859	55.6	1865	27.7	1865	21.2	1865	
Celata et al. [18]	33.0	43	45.4	43	13.6	41	18.0	43	
Celata et al. [19]	17.9	88	22.9	44	15.2	88	15.5	88	
Celata et al. [21]	23.3	48	16.8	48	15.6	48	19.3	48	
Celata et al. [21]	22.3	7	20.2	7	15.1	7	18.0	7	
Celata et al. [20]	22.5	60	23.1	60	21.1	60	17.4	60	
Inasaka-Nariai [15]	20.4	29	38.3	29	12.1	29	10.6	29	
Nariai et al. [16]	44.6	95	109.1	95	45.3	95	29.0	95	
Boyd [12–14]	84.0	8	30.3	10	29.3	10	10.9	10	
Achilli et al. [17]	14.8	35	22.2	35	21.2	35	30.7	35	
Gambill-Greene [22]	43.4	7	111.2	7	89.0	7	14.4	7	
Vandervort et al. [23]	33.5	210	22.2	210	18.3	210	14.6	210	
Loosmore-Skinner [24]	27.0	202	33.4	202	30.6	202	19.6	20	
Ornatskii–Vinyarskii [25]	22.7	125	21.3	125	18.5	125	24.8	125	
Ornatskii-Kichigan [26]	27.9	117	37.9	117	25.1	117	26.1	117	
Ornatskii [27]	24.9	68	20.8	68	21.0	68	26.8	68	
Knoebel et al. [28]	94.3	376	84.0	376	27.6	376	10.8	376	
Mirshak et al. [29]	130.0	52	98.3	56	34.7	56	15.2	56	
Babcock [30]	59.9	57	34.9	57	18.6	57	25.8	57	
Burck-Hufschmidt [31]	56.1	143	13.0	143	36.1	143	37.2	143	
Mayersak et al. [32]	7.1	1	2.6	1	74.9	1	13.4	1	
Schaefer–Jack [33]	65.6	2	14.9	2	26.8	2	27.8	2	
Thorgerson [34]	117.5	42	80.7	42	36.3	42	8.9	42	
Zeigarnik et al. [35]	54.3	21	12.3	21	17.8	21	14.5	21	
Gambill et al. [36]	47.7	23	81.5	23	80.1	23	27.6	23	

Table 3. Calculated r.m.s. errors for the correlations

Model	Ka	itto	Lee-Mudawar		Weisman-Ileslamlou	
CHF Data	r.m.s.		r.m.s.		r.m.s.	
[ref.]	[%]	Points	[%]	Points	[%]	Points
Total	24.5	915	155.1	1847	82.6	1864
Celata et al. [18]	16.8	26	148.6	43	80.3	43
Celata et al. [19]	21.2	81	83.9	88	27.5	88
Celata et al. [21]	14.6	42	76.8	48	29.1	48
Celata et al. [21]	27.8	5	91.5	7	22.4	7
Celata et al. [20]	11.9	58	80.6	60	22.0	60
Inasaka–Nariai [15]	25.5	15	138.8	29	58.2	29
Nariai et al. [16]	32.4	17	338.3	95	95.3	95
Boyd [12–14]	18.0	6	69.3	10	95.5	10
Achilli et al. [17]	19.1	35	47.4	35	18.9	35
Gambill-Greene [22]	21.0	7	282.0	7	18.2	7
Vandervort et al. [23]	19.2	155	115.2	210	23.0	210
Loosmore-Skinner [24]	19.7	12	151.2	202	63.7	202
Ornatskii-Vinyarskii [25]	26.9	121	61.0	125	29.5	125
Ornatskii-Kichigan [26]	42.1	31	126.9	117	58.2	117
Ornatskii [27]	30.0	61	67.2	68	31.0	68
Knoebel et al. [28]	31.4	107	193.6	376	130.2	375
Mirshak et al. [29]	35.3	6	205.2	56	182.9	56
Babcock [30]	19.2	32	115.6	57	61.8	57
Burck Hufschmidt [31]	17.3	57	44.4	125	54.5	143
Mayersak et al. [32]	93.2	1	11.6	1	21.9	1
Schaefer–Jack [33]	32.9	2	13.7	2	60.0	2
Thorgerson [34]	37.5	8	177.7	42	142.7	42
Zeigarnik et al. [35]	8.58	11	55.0	21	57.3	21
Gambill et al. [36]	28.7	23	220.0	23	42.8	23

Table 4. Calculated r.m.s. errors for the models

sure or the tube diameter or length. A substantial absence of systematic deviations is presented by the modified-Tong correlation.

As far as model predictions are concentred, the Katto model turns out to be the best one in the present calculations, with an even distribution of points within the error band. The model was assessed only on data [12, 15, 18] among those used in present calculations and works very well with the other datasets. A small systematic effect (underprediction) is shown by the channel length (Fig. 12) for very short channel data. It must be considered, however, that the Katto model is unable to give a prediction for about one half of the available experimental data, as stated above. This is, unfortunately, a limiting aspect of this interesting predicting tool. The Weisman and Ileslamlou model provides predictions affected by a systematic error, even though, globally, some of them lie within  $\pm 25\%$ . The Lee and Mudawar model gives a general inadequacy in the prediction of available low pressure data. As expected, both the Weisman and Ileslamlou and the Lee and Mudawar models show a systematic dependence on thermal-hydraulic and geometric parameters (Figs. 10 and 11), even though the Lee and Mudawar model shows a successful prediction of high pressure data. It must be pointed out again that both these two models were assessed, and therefore recommended by respective authors, in their proposed version, in a range of pressure above 7.0 MPa, and then outside the pressure range of used data.

# CONCLUDING REMARKS

With the aim of establishing the bounds of validity of existing predictive tools for the calculation of the CHF in subcooled flow boiling, four correlations (Levy, Westinghouse, modified-Tong and Tong-75) and three models (Weisman and Ileslamlou, Lee and Mudawar and Katto) have been statistically analysed using twenty-four data sets available in literature for a total of 1865 data points, in wide ranges of operating conditions typical of fusion reactor thermal-hydraulics requirements (0.1mm; 0.1 < L < 0.61 m;  $2 < G < 90.0 \text{ Mg m}^{-2} \text{ s}^{-1}$ ;  $90 < \Delta T_{\text{sub.in}} < 230$  K). Statistics and r.m.s. errors have been calculated for each predictive tool and each data-set, and comparisons of correlations and model predictions with experimental data have been shown. Correlations and models have been characterized in terms of thermal hydraulic (mass flux and pressure) and geometric (channel diameter and length) parameters to ascertain possible systematic effects in predictive tools performances.

Among the correlations, a very good agreement with experimental data is shown by the modified-Tong correlation (modified on the basis of ENEA data [18– 21]) characterized by a very good statistics (76.5% of predictions are within  $\pm 25\%$ ) and by an r.m.s. error of 21.2%. The wide ranges of operating conditions which the present calculations have been done on, allow us to give this correlation (which has the advantage of being a very simple correlation) a good





FIG. 5. Calculated vs experimental CHF for the three models. See Fig. 4 for legend.















reliability for the prediction of the CHF in subcooled flow boiling. Other correlations show a fairly good agreement with many experimental data, but are unreliable when used very far from the recommended ranges of application.

Among the models, a very good prediction of experimental data is provided by the Katto model, which was proposed by the author on the basis of few present data ([12, 15, 18]). The model is characterized by good statistics (72.3% of predictions are within  $\pm 25\%$ ) and by a r.m.s. error of 24.5%. A limit of the Katto model is represented by the fact that it is not able to calculate all the data points. It fails for 950 points out of 1865 (50.9%) that are therefore discarded. This is due to the calculation procedure of the Katto model that requires a void fraction in the boiling layer less than 0.7. This is the limit considered by the author for the validity of the assumption of homogeneous flow in the two-phase boundary layer. The Katto model, although mechanistic in nature, shows the necessity, like the other two models, of empirical parameters introduced in the mathematical description of the dynamics of the bubbles that must be derived from experiments. It is therefore still necessary to accomplish a full understanding of the phenomenon to propose a realistic and pure mechanistic model description.

Acknowledgements—The authors wish to thank Prof. A. E. Bergles, Dr G. P. Gaspari, Dr M. K. Jensen and Prof. H. Nariai who provided their experimental data in a form useful for calculations, and gave valuable suggestions for the present work. Thanks are also due to Prof. Y. Katto who provided the manuscript of his model prior to publication, and Prof. I. Mudawar for much advice for the present analysis. The authors are grateful to Mrs. A. Moroni who took care of the editing of the paper.

#### REFERENCES

- G. P. Celata, A review of recent experiments and predictions aspects of burnout at very high heat flux, *Proceedings of the International Conference on Multiphase Flows '91-Tsukuba*, Vol. 3, pp. 31-40 (1991).
- 2. G. P. Celata, Fundamental studies of CHF in highly subcooled water flow boiling—ENEA experimental facility and CHF results, *Proceedings of the Specialists' Workshop on Divertor Plates Thermal-Hydraulics*, San Francisco (1991).
- R. D. Boyd, Subcooled flow boiling Critical Heat Flux (CHF) and its application to fusion energy components. Part I: A review of fundamentals of CHF and related data base, *Fusion Technol.* 7, 7–30 (1985).
- R. D. Boyd, Subcooled flow boiling Critical Heat Flux (CHF) and its application to fusion energy components. Part II: A review of microconvective, experimental, and correlational aspects, *Fusion Technol.* 7, 31–52 (1985).
- S. Levy, Prediction of the Critical Heat Flux in forced convection flow, *General Electric Report GEAP*-3961 (1962).
- L. S. Tong, H. B. Currin and A. G. Thorp, An evaluation of the departure from nucleate boiling in bundles of reactor fuel rods, *Nuclear Sci. Engng* 33, 7–15 (1968).
- 7. L. S. Tong, Boundary-layer analysis of the flow boiling crisis, *Int. J. Heat Mass Transfer* **11**, 1208–1211 (1969).
- 8. L. S. Tong, A phenomenological study of Critical Heat Flux, ASME Paper 75-HT-68 (1968).

- J. Weisman and S. Ileslamlou, A phenomenological model for prediction of Critical Heat Flux under highly subcooled conditions, *Fusion Technol.* 13, 654-659 (1988) (Corrigendum in *Fusion Technol.* 15, 1463 (1989)).
- C. H. Lee and I. Mudawar, A mechanistic Critical Heat Flux model for subcooled flow boiling based on local bulk flow conditions, *Int. J. Multiphase Flow* 14, 711 728 (1988).
- Y. Katto, A prediction model of subcooled water flow boiling CHF for pressure in the range 0.1 20.0 MPa, *Int. J. Heat Mass Transfer* 35, 1115–1123 (1992).
- R. D. Boyd, Subcooled water flow boiling experiments under uniform high flux conditions, *Fusion Technol.* 13, 131-142 (1988).
- R. D. Boyd, Subcooled water flow boiling at 1.66 MPa under uniform high heat flux conditions, *Proceedings of* the ASME Winter Annual Meeting, HTD, Vol. 119, pp. 9–15 (1989).
- R. D. Boyd, Subcooled water flow boiling transition and the L/D effect on CHF for a horizontal uniformly heated tube, Fusion Technol. 18, 317–324 (1990).
- F. Insaka and H. Nariai, Critical heat flux of subcooled flow boiling with water, *Proceedings of the NURETH*-4, Vol. 1, pp. 115–120 (1989).
- H. Nariai, F. Inasaka and T. Shimura, Critical Heat Flux of subcooled flow boiling in narrow tube, *Proceedings* of the ASME JSME Thermal Engineering Joint Conference, pp. 455–462 (1987).
- A. Achilli, G. Cattadori and G. P. Gaspari, Subcooled burnout in uniformly and non-uniformly heated tubes, *European Two-phase Flow Group Meeting*, Paper C2, Stockholm, June 1-3, 1992.
- G. P. Celata, M. Cumo and A. Mariani, Subcooled water flow boiling CHF with very high heat fluxes, *Rerue Générale de Thermique* 362, 106-114 (1991).
- G. P. Celata, M. Cumo and A. Mariani, Burnout in highly subcooled flow boiling in small diameter tubes. *Int. J. Heat Mass Transfer* 36, 1269–1285 (1991).
- G. P. Celata, M. Cumo and A. Mariani, CHF in highly subcooled flow boiling with and without turbulence promoters. *European Two-Phase Flow group Meeting*, Paper C1, Stockholm, June 1–3 (1992).
- 21. G. P. Celata, M. Cumo and A. Mariani, *ENEA Report* (not published).
- W. R. Gambill and N. D. Greene, Boiling burnout with water in vortex flow, *Chem. Engng Prog.* 54(10), 68–76 (1958).
- C. L. Vandervort, A. E. Bergles and M. K. Jensen, The ultimate limits of forced convective subcooled boiling heat transfer, *RPI Report* HTL-9 (1990).
- C. S. Loosmore and B. C. Skinner, Subcooled Critical Heat Flux for water in round tube, S.M. Thesis, Massachusetts Institute of Technology, Cambridge, Massachusetts (1965).
- A. P. Ornatskii and L. S. Vinyarskii, Heat transfer crisis in a forced flow of underheated water in small bore tubes, *Teplofizika Vysokikh Temperatur (High Temperature)* 3(3), 444–451 (1964).
- A. P. Ornatskii and A. M. Kichigan, Critical thermal loads during the boiling of subcooled water in small diameter tubes, *Teploenergetika* 6, 75–79 (1962).
- 27. A. P. Ornatskii, The influence of length and tube diameter on critical heat flux for water with forced convection and subcooling, *Teploenergetika* **4**, 67–69 (1960).
- D. H. Knoebel, S. D. Harris, Jr, B. Crain and R. M. Biderman, Forced-convection subcooled critical heat flux, DP-1306, E.I. Dupont de Nemours and Company (1973).
- S. Mirshak, W. S. Durant and R. H. Towell, Heat flux at burnout, DP-355, E.I. Dupont de Nemours and Company (1959).

- D. F. Babcock, Heavy water moderated power reactors, DP-725, E.I. Dupont de Nemours and Company (1962).
- 31. E. Burck and W. Hufschmidt, EUR-2432 d (in German), EURATOM (1965).
- J. Mayersak, S. D. Raezer and E. A. Bunt, Confirmation of Gambill-Green straight flow burnout heat flux equation at higher flow velocity, J. Heat Transfer 86, 420– 425 (1964).
- J. W. Schaefer and J. R. Jack, Investigation of forcedconvection nucleate-boiling of water for nozzle cooling at very high flux, *Technical Note* D-1214, NASA (1962).
- 34. E. J. Thorgerson, Hydrodynamic aspect of the Critical Heat Flux in subcooled convection boiling, Ph.D. Thesis, University of South Carolina (1969).
- Yu. A. Zeigarnik, N. P. Privalov and A. I. Klimov, Critical Heat Flux with boiling of subcooled water in rectangular-channel with one-sided supply of heat, *Thermal Engng* 28(1), 40-42 (1981).
- 36. W. R. Gambill, R. D. Bundy and R. W. Wansbrough, Heat transfer, burnout, and pressure drop for water in swirl flow through tubes with internal twisted tapes, *Chem Engng Prog. Symp. Ser.* 57(32), 127–132 (1961).
- A. E. Bergles, J. G. Collier, J. M. Delhaye, G. F. Hewitt and F. Mayinger, *Two-Phase Flow and Heat Transfer in* the Power and Process Industries, pp. 226–255, Hemisphere, New York (1981).
- J. G. Collier, Convective Boiling and Condensation, pp. 144–177. McGraw-Hill, New York (1981).
- Y. Y. Hsu and R. W. Graham, Transport Processes in Boiling and Two-Phase Systems, pp. 217–232. American Nuclear Society (1986).

- G. P. Celata, Critical analysis of correlations and models for highly subcooled CHF predictions, *Proceedings of* the Specialists' Workshop on Divertor Plates Thermal-Hydraulics, San Francisco (1991).
- H. Nariai and F. Inasaka, Remarks on subcooled flow boiling CHF correlations, *Proceedings of the Specialists'* Workshop on Divertor Plates Thermal-Hydraulics, San Francisco (1991).
- 42. J. R. S. Thom, W. W. Walker, T. A. Fallon and G. F. S. Reising, Boiling in subcooled water during flow up heated tubes of annuli, Paper 6 presented at the Symposium on Boiling Heat Transfer in Steam Generating Units and Heat Exchangers, Manchester, 1MechE, September 15–16 (1965).
- J. Weisman and B. S. Pei, Prediction of the Critical Heat Flux in flow boiling at intermediate qualities, *Int. J. Heat* Mass Transfer 26, 1463-1477 (1983).
- 44. Y. Haramura and Y. Katto, A new hydrodynamic model of critical heat flux, applicable widely to both pool and forced convection boiling on submerged bodies in saturated liquids, Int. J. Heat Mass Transfer 26, 389– 399 (1983).
- 45. Heat Mass Transfer Section, Scientific Council, Academy of Sciences, U.S.S.R., Tabular data for calculating burnout when boiling water in uniformly heated round tubes, *Thermal Engng* 23, 77–79 (1972).
- 46. B. Thompson and R. V. Macbeth, Boiling water heat transfer burnout in uniformly heated round tubes: a compilation of world data with accurate correlations, U.K.A.E.A., AEEW-R 359 (1964).